Thomas Mogren

Manager - Engineering Services



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August 30, 2011

U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

ATTENTION: Document Control Desk

SUBJECT:

R.E. Ginna Nuclear Power Plant

Docket No. 50-244

Transmittal of 2011 Owner's Activity Report for the

R.E. Ginna Nuclear Power Plant

Enclosed is a copy of the R.E. Ginna Owner's Activity Report for the refueling outage conducted in 2011. This report is submitted as specified by ASME Code Section XI.

There are no regulatory commitments contained in this letter. Should you have questions regarding this matter, please contact Thomas Harding (585) 771-5219, or Thomas.HardingJr@cengllc.com.

Very truly yours,

Thomas Mogren

Attachment:

(1) FIFTH INSERVICE INSPECTION (ISI) INTERVAL and SECOND CONTAINMENT INSPECTION INTERVAL, FIRST INSPECTION

PERIOD, 2011 OWNER'S ACTIVITY REPORT FOR RFO-35 INSERVICE

EXAMINATIONS, (April 23, 2011 through June 9, 2011)

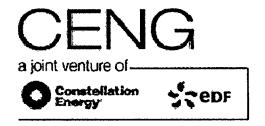
cc:

W.M. Dean, NRC

D.V. Pickett, NRC

Resident Inspector, NRC (Ginna)

4047 MRR



FIFTH INSERVICE INSPECTION (ISI) INTERVAL and SECOND CONTAINMENT INSPECTION INTERVAL

FIRST INSPECTION PERIOD 2011 OWNER'S ACTIVITY REPORT FOR **RFO-35 INSERVICE EXAMINATIONS** (April 23, 2011 through June 9, 2011)

For

R. E. Ginna Nuclear Power Plant 1503 Lake Road Ontario, NY 14519

Commercial Service Date:			June 1, 1970
NRC Docket Number:			50-244
Revision Number:	00	Date:	August 11, 2011

Prepared by: Frank & Klepacki	8/11/2011
Frank A. Klepacki – Site ISI Program Owner – R E Ginna	
Reviewed by: 250 Wall 8 11 11 Bernie C. Rudell – Principal Engineer – CENG Fleet Engineer	ing Programs
Approved by: Jak Wells - General Supervisor - Engineering Programs - R.E.	Ginna



5th Interval Inservice Inspection and 2nd Interval Containment Inservice Inspection
2011 OWNER'S ACTIVITY REPORT FOR INSERVICE EXAMINATIONS

Rev. 00

Date: 08/11/2011

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5th Interval Inservice Inspection and 2nd Interval Containment Inservice Inspection 2011 OWNER'S ACTIVITY REPORT FOR INSERVICE EXAMINATIONS

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ABSTRACT

This Owner's Activity Report (OAR) summarizes the results of the R. E. Ginna Nuclear Power Plant (GNPP), Unit 1, American Society of Mechanical Engineers (ASME) Section XI Inservice Inspection Examinations performed during the 2011 Refueling Outage (RFO), from April 23, 2011 through June 9, 2011. The OAR is for the Fifth Interval Inservice Inspection Program, First Period and the Second Interval Containment Inservice Inspection Program, First Period. This report also covers associated Reactor Pressure Vessel (RPV) Fourth Interval examinations that were deferred in accordance with the Fourth Interval Relief Request Numbers 18, 19, 20 and 21. The RPV examinations were performed in accordance with ASME Section XI Code, 1995 Edition with the 1996 Addenda. The First Interval Containment Inservice Inspection Program was extended to align with the ISI Program interval in accordance with the Fourth Interval Inservice Inspection Program Relief Request Number 22.

As an alternate to the requirements of the ASME, Boiler and Pressure Vessel Code (B&PVC), Section XI, Article IWA-6000, GNPP has implemented ASME Code Case N-532-4, "Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000, Section XI, Division 1". ASME Code Case N-532-4 is approved for use in Regulatory Guide 1.147. This Report complies with the requirements of Code Case N-532-4.

OWNER'S ACTIVITY REPORTS - FORM OAR-1

Owner's Activity Reports Form OAR-1's were prepared and certified upon completion of the GNPP Unit 1, 2011 refueling outage RFO-35 examination activity in accordance with the following inspection plans and schedules:

- Inservice Inspection Program Plan(s) for the Fifth Inspection Interval for GNPP Unit 1
- Long Term Plan for GNPP, Unit 1
- ISI Program Plan ASME XI, IWE/IWL Second Inspection Interval

In addition to the Owner's Activity Report, Form OAR-1, the following Tables are also included:

- a. Items with Flaws or Relevant Conditions that Require Evaluation for Continued Service, Table 1, provides a listing of item(s) with inservice flaws or relevant conditions that required evaluation to determine acceptability for continued service, whether or not the flaw or relevant condition was discovered during a scheduled examination or test, and provides such information as Examination Category, Examination Item Number, Item Description, Flaw Characterization (as required by IWA-3300).
- b. Abstract of Repairs, Replacements or Corrective Measures Required for Continued Service, Table 2, identifies those items containing flaws or relevant conditions that exceeded the ASME Section XI acceptance criteria IWX-3000. In the case of supports, the acceptance criteria is IWF-3000. The discovery of the flaw or relevant condition that necessitated the repair, replacement or corrective measure, may not have resulted from an examination or test required by Section XI Code. Table 2 provides information such as Code Class, whether the item is a repair, replacement or corrective action, item description, description of work, if the flaw or relevant condition was found during a scheduled Section XI Examination or test, date completed and the repair replacement plan number.



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FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number	GNPP-OAR-ISI-RPV-2011			
Plant <u>R. E.</u>	Ginna Nuclear Power Plant, 1503 Lake Road, Ontario, New York 14519 (Name and Address of Plant)			
Unit No1_	Commercial Service Date <u>June 1, 1970</u> Refueling Outage Number <u>RFO-35</u>			
Current Inspection	Interval Fourth Inservice Inspection Interval - January 1, 2000 to June 9, 2011. (1st, 2nd, 3rd, 4th, Other)			
Current Inspection	PeriodThird Inservice Inspection PeriodJanuary 1, 2007 to June 9, 2011 (1st, 2nd, 3rd)			
Edition and Adder	nda of Section XI applicable to the Inspection Plan1995 Edition, 1996 Addenda			
Date and Revision	of Inspection Plan <u>4 October 2010, Revision 1</u>			
Edition and Addenda	a of ASME Section XI applicable to repairs/replacements, if different than the Inspection Plan N/A			
Code Cases used:_	N-532-4, N-695,			
the ASME Code, conform to the re	the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by Section XI; and (c) the repair/replacement activities and evaluations supporting completion of 2011 RFO (RFO-35) equirements of Section XI. Sequirements of Section XI. Thorization No. N/A Expiration Date N/A A Klepacki Full Apacki Date S/11/2011 SI Program Owner)			
	CERTIFICATE OF INSERVICE INSPECTION			
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by Hartford Steam Boiler (HSBCT) of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection. Bruce Sawyer Commissions NB10496, A. N. I; New York 4077 Inspector's Signature National Board, State, Province, and Endorsements Date 8/11/20//				



5th Interval Inservice Inspection and 2nd Interval Containment Inservice Inspection 2011 OWNER'S ACTIVITY REPORT FOR INSERVICE EXAMINATIONS Rev. 00

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FORM OAR-1 OWNER'S ACTIVITY REPORT

As required by the provisions of the ASME Code Case N-532-4						
Report Number	GNPP-OAR-ISI-2011					
Plant R. E, Ginna Nuclear Power Plant, 1503 Lake Road, Ontario, New York 14519 (Name and Address of Plant)						
Unit No. 1 Commercial Service Date <u>June 1, 1970</u> Refueling Outage Number <u>RFO-35</u>						
Current Inspection Interv	Current Inspection Interval <u>Fifth Inservice Inspection Interval</u> - <u>January 1, 2010 to December 31, 2019.</u> (1st, 2nd, 3rd, 4th, Other)					
Current Inspection Perio	Current Inspection Period First Inservice Inspection Period – January 1, 2010 to December 31, 2012 (1st, 2nd, 3rd)					
Edition and Addenda of	Edition and Addenda of Section XI applicable to the Inspection Plan2004 Edition, No Addenda					
Date and Revision of Inspection Plan <u>4 October 2010, Revision 1</u>						
Edition and Addenda of ASME Section XI applicable to repairs/replacements, if different than the Inspection Plan N/A						
Code Cases used: N-532-4, N-648-1, N-706, N-716, N-722, N-729-1						
			The traction			
	CERTIFICATE OF	CONFORMANCE				
I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting completion of 2011 RFO (RFO-35) conform to the requirements of Section XI.						
Certificate of Authorizat	ion No. N/A (If applicable)	Expiration Date	N/A			
Signed <u>Frank A Kler</u> (Site ISI Prog	packi Frank & Klase	Date 8	7/11/2011			

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by Hartford Steam Boiler (HSBCT) of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection. Bruce Sawyer Commissions NB10496, A, N, I; New York 4077 Inspector's Signature Commissions NB10496, A, N, I; New York 4077 National Board, State, Province, and Endorsements



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FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number _		GNPP-OA	R-IWE/L-2011		
Plant <u>RE</u>	Plant R E Ginna Nuclear Power Plant, 1503 Lake Road, Ontario, New York 14519 (Name and Address of Plant)				
Jnit No1_	Commercial Se	rvice Date <u>June</u>	1, 1970	Refueling Outage	Number <u>RFO-35</u>
Current Inspection	n Interval <u>S</u>		Inspection Interval and, 3rd, 4th, Other)	– January 1,2010 to	December 31, 2019
Current Inspection	n Period <u>Fir</u>		<u>ion Period – January</u> (1st, 2nd, 3rd)	1, 2010 to Decemb	er 31, 2012
Edition and Adde	nda of Section X	l applicable to the li	nspection Plans	2004 Edition, No A	ddenda
Date and Revision	n of Inspection P	lans 29 Dece	ember 2009, Revision	10	
Edition and Addend	a of ASME Section	ı XI applicable to repa	irs/replacements, if diff	erent than the Inspection	on Plan <u>N/A</u>
Code Cases used:_	N-532-4, N-6	86, N-739, N-753, N-6			· · · · · · · · · · · · · · · · · · ·
			(if applicable)		
I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting completion of _2011 RFO_(RFO-35) conform to the requirements of Section XI. Certificate of Authorization No					
(Site	ISI Program Owne	1)			
		CERTIFICAT	E OF INSERVICE	INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by Hartford Steam Boiler (HSBCT) of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI. By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation/described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection. Bruce Sawyer Commissions NB10496, A. N. I; New York 4077 Inspector's Signature National Board, State, Province, and Endorsements Date Still Joil					



5th Interval Inservice Inspection and 2nd Interval Containment Inservice Inspection 2011 OWNER'S ACTIVITY REPORT FOR INSERVICE EXAMINATIONS

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TABLE 1 ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRE EVALUATION FOR CONTINUED SERVICE

EXAMINATION CATEGORY AND ITEM NUMBER	ITEM DESCRIPTION	EVALUATION DESCRIPTION
B-P, B15.80 Code Case N-722 ISI Fourth Interval Relief Request Number 24 ISI Summary Number I411111	Reactor Pressure Vessel (RPV) Bottom Mounted Instrumentation, Penetration #31 (A86/J-12), Ultrasonic (UT) examination was performed as required by Relief Request Number 24 and an indication identified that was determined to be rejectable in accordance with IWB-3000.	Engineering Analytical Evaluation was performed to satisfy the requirements of IWB-3660 demonstrates acceptance for a one additional cycle. A second Analytical Evaluation was performed by Westinghouse and justified operation for an additional 40 years until May 2051.
B-N-3, B13.70 ISI Summary Number I000800	Reactor Pressure Vessel (RPV) Core Support Structure Baffle Bolt Replacement Baseline VT examination notes that three (3) baffle bolt locations no longer have a bolt there. Bolt replacement attempts experienced problems resulting in the elimination of three bolts.	Engineering Analytical Evaluation was performed under ECP-10-000422. The evaluation shows the condition is acceptable until the next 10-Year ISI RPV Core Support Structure Examination.
Reactor Vessel Internals Program LR-RVI-PROGPLAN Revision 2, Attachment D Baffle-Former Assembly Baffle Former Bolts.	Reactor Vessel Internals Program UT examinations of Baffle Former Bolts identified one bolt with a rejectable indication.	Engineering Analytical Evaluation was performed under ECP-10-000422. This plant specific evaluation shows the baffle-former bolting condition is acceptable for 10 additional years of operation when re-examination or replacement is planned.



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TABLE 2 ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

CODE	ITEM	DESCRIPTION	DATE	REPAIR/REPLACEMENT
CLASS	DESCRIPTION	OF WORK	COMPLETED	PLAN NUMBER
2	AFU-227, Mechanical Snubber, PSA-3	Snubber Functional Test Failure - Replaced Mechanical Snubber on component support AFU-227.	June 2011	GORR 35-076 (WO C90797825)
1	Reactor Coolant Pump (RCP) "A", Stud # 8, ASME Category B-G-1 Item Number B6.180 ISI Summary Number I600049.	Ultrasonic examination performed on all 24 studs identified an indication in one stud (#8) that could not be sized for evaluation. Therefore, stud #8 was rejected and replaced with new stud #25. Stud #25 was a like-for-like replacement; UT baseline examination was performed and acceptable. UT bolting examination on the remaining 23 RCP "A" studs were acceptable. An expansion was performed to the 24 RCP "B" studs, UT examined and all acceptable.	June 2011	GORR 35-090 (WO C20900841)



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END OF INSERVICE INSPECTION OWNER'S ACTIVITY REPORT FOR

2011 REFUELING OUTAGE (RFO-35)